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PRELIMINARY REPORT ON FISSION CAUSED BY FISSION NEUTRONS

By

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It appeared to be important to determine whether fission can be caused in uranium by fission neutrons and if so to learn something about the cross-section of this process.

This process may play at important role in a chain reaction in a system in which spheres of uranium are imbedded in a large mass of graphite. In such a system thermal neutrons diffuse from the graphite into the uranium sphere and lead to the emission of fast fission neutrons. The fast neutrons thus produced will cause a certain number of fission processes in the same uranium sphere from which they originate and thus produce secondary fast fission neutrons. Some of these secondary neutrons will again cause fission in the uranium sphere from which they originate and lead to tertiary fission neutrons, etc. This process might increase by perhaps to much af a factor of $\beta = 1.25$ the number of fast neutrons emitted by uranium per thermal neutron absorbed by uranium. The magnitude of the factor β depends primarily upon certain nuclear properties of uranium and in the first approximation on the product

of (V-1)

where $\mathbf{6}$ is the cross-section of uranium for fast fission neutrons and $\mathbf{1}$ is the number of neutrons emitted per fission process. The magnitude of the factor β also depends on the cross-section of uranium for inelastic collisions which slow down fast fission neutrons below the fission threshold of U238. This slowing-down phenomenon of uranium has been studied by Szilard and Zinn and forms the subject of another report.

Secondarily, the magnitude of the factor ß depends on the size and density of the uranium sphere and also on the presence in the uranium sphere of oxygen or other elements which slow down the neutrons by means of elastic collisions. It may very well be that, due to the process which is the subject of this report, there may be a ten per cent difference in the number of fast neutrons emitted by a uranium sphere per thermal neutron absorbed by the uranium sphere if we change over from uranium oxide at a density of about 4 to 6 gm./cc. to uranium metal of density 18 to 20 gm./cc. The experiment which we performed has the purpose of measuring the quantity

The principle of the experiment is illustrated in the enclosed diagram which shows the experimental arrangement. In this diagram Be is a beryllium block which serves as a source of photo neutrons and which is placed in the axis of a cylindrical paraffin block. Ra represents about two grams of radium which are placed in the center of the beryllium block. A cadmium diaphragm A leaves a circular opening free through which thermal neutrons can emerge from the paraffin and can enter in the 2.

absence of the cadmium screen B. These thermal neutrons can enter the cylindrical box U which contains uranium in the amount of about 25 grams per square cm. A spherical ionization chamber Awhich is coated with a thick layer of uranium and has a uranium-coated surface of about 800 cm. square is used to record fission which takes place in the uranium coating of this chamber. This fission chamber is protected from the action of thermal neutrons by thick devers of boron carbide, and Eis a boron carbide screen which can be removed if it is desired to admit thermal neutrons to the fission chamber.

The basic experiment which we performed is the pla following: With the boron screen E and the cadmium screen B/ in position, the fission chamber registers a background of about .5 fissions per minute. Thermal neutrons which come from the circular opening in the center of the cadmium diaphragm A are prevented by the cadmium screen of from reaching the uranium during this control experiment. If the cadmium screen is now brought from the position into the position , thermal neutrons are admitted to the uranium, will cause fission in the uranium and will lead to the emission of fast fission neutrons. considerable fraction of these neutrons passes through the unu illed loox U whole thickness of the uranium layer and through the boron screen ianisah an E and will cause fission in the fission chamber D. By changing the position of the cadmium screen from position B to position B, we obtained a fission count which was more than double of the background count, the difference corresponding to about .7% counts per minute. A preliminary estimate of the quantities involved

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leads to the conclusion that the observed effect corresponds to

a value of about

GEV= 1.3x10 cm2

Taking for V the value of 2.6 reported by Zinn and Szilard, $\sigma_{p} = 0.5 \times 10^{-24}; \quad \sigma_{p}(V-1) = 0.8 \times 10^{-24}$ we obtain

It has to be emphasied that this is a preliminary result, and that the experiments will be repeated, and that an attempt will be made to determine the value more accurately than it was possible to do in the first experiment. In the following we describe two methods which were used in interpreting the observed The following designations will be used: effect.

Ff is the fission count which we obtain from the fast neutrons when we remove the cadmium screen Bar M

- Fth is the fission count which we obtain in the chamber (in the absence of the uranium, the cadmium screen B/ or k and the boron screen E) due to the action of thermal neutrons which emerge from the window in the center of cadmium diaphragm A, and which can be cut off by the cadmium screen in the position \$.
- N is the number of thermal neutrons which emerge from the circular window in the center of the cadmium diaphragm
- a is infraction of these thermal neutrons which are absorbed by uranium in the look U.
- 1ε is the fraction of the fast neutrons which are emitted by uranium and which are prevented from reaching the laple ionization chamber by scattering either to uranium or by Bort in the boron screen E. which time Surland Phys. Rev. 1939

A.

por unit solod $h/2\pi$ is the number of thermal neutrons emitted from the paraffin wall in the circular opening in the direction per unit solid angle.

U is the number of uranium atoms which are exposed to the thermal neutrons.

P is the thermal neutron density of the uranium.

- r is the distance of a volume element of uranium in the Box \mathcal{U} from a surface element of the uranium layer in the spherical ionization chamber.
- 6th (tilssion) is the fission cross-section of uranium for thermal neutrons.
- oth is the total absorption cross-section of uranium for thermal neutrons.
- is a thermal neutron density XXXXX/ the location of the uranium surface of the spherical ionization chamber in the absence of the uranium, box k', the cadmium screens B and ξ' and the boron screen E.

The main quantities which we measure are F_{th} and F_{f} . One of the methods employed makes use of the fact that we may write:

THE = NG O H (fision) mth-FR = NCE Orthission (OPV) nt OFV = FLOTH 26 min The

In these formulae N_{th}(5) is the number of uranium atoms which is effective in producing an impulse in the chamber if excited by thermal neutrons, and N(5) is the number of uranium atoms which is effective in producing an impulse in the chamber if excited by fast fission neutrons. We have actually assumed that these two numbers are equal, i.e., that the range of the fission particles is about the same when the fission is due to thermal neutrons as it is when the fission is due to fast fission neutrons. From these two equations, we obtain:

Our experiments gave for $F_{\rm th}$ a value of 43 counts per minute (cadmium difference), and for $F_{\rm f}$ we obtained from 25 fifteen-minute readings with the cadmium screen at the position H and K, respectively, a value of $H = F_{\rm f}$

above a background count of . 6 per minute.

nth

c, the fraction of thermal neutrons which are absorbed by the uranium in the box, was estimated to be about 0.5 The uranium box contains about 27.7 gms./sq.cm. of uranium carbide or about 25.2 gms./sq.cm. of uranium. Meglecting of the thermal neutrons, of the thermal neutrons passing through the uranium box parallel to the axis of the arrangement, a fraction of .31 would be absorbed, and a fraction of .687 would be transmitted by the uranium box. By measuring the thermal neutron density with a vanadium indicator in front and right behind the uranium box, we find that the thermal neutron density for by a factor of 8.in our arrangement. Part of this drop is due to the scattering of thermal neutrons

6.

in the uranium box, and a very rough estimate leads us to believe that actually about .5 of the neutrons entering the uranium box are absorbed.

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7.

For b we take the value of

b = 2.53

which corresponds to distribution of an angular distribution $4 \cos f + \sqrt{3} \times \cos^2 f$ for the spatial distribution of thermal neutrons leaving the paraffin window G inside the cadmium diaphragm A.

For ε we took the value of . by estimating the effect of the scattering in the boron carbide screen E containing 3 gms./sq.cm. of boron carbide and the effect of scattering of the uranium carbide in the uranium box.

For $\overrightarrow{7}$ we took the value of $\binom{13}{74} = 0.87$ For 5 - 4 we took the value of 5.9 x 10-24 sq.cm. With these values we obtained from (1) $0 \neq V = 1.24$

The second method employed makes use of the fact that we may write

(2)OFV=47 So (TR) 1 = FK nth

We estimated \int inside the uranium, and \int_{o} , the thermal neutron density in the space occupied by the ionization chamber D by observing the activity of a vanadium foil in front

of the window \mathcal{C} on both sides of the uranium box and fin the absence of the uranium box, the cadmium screens at H or K and the boron screen E with a vanadium foil taking the place of the ionization chamber D. Assuming an exponential full of the thermal neutron density inside the uranium box, we then find for $4\pi \int_{\mathcal{O}}^{\infty} \mathcal{O}(\mathcal{O}) = (4\pi \eta^2)$ $\mathcal{O}(\mathcal{O}) = 250$ For the number of uranium atoms in front of the window G we take the number of U = 4.63 to corresponding to 1830 gms. of uranium. For ε we again take the value of $\mathcal{O}(\mathcal{O})$ With these values we obtain from (2)

QV = 1.28 (2)R)

